

8. Effects of 135Xe

- The xenon isotope ¹³⁵Xe plays an important role in any power reactor.
- It has a very large absorption cross section for thermal neutrons and represents therefore a considerable load on the chain reaction.
- The ¹³⁵Xe concentration has an impact on the power distribution,
- and in turn is affected by the power distribution, by changes in power, and by movements of reactivity devices.



- ◆ The ¹³⁵Xe/¹³⁵I kinetics are shown schematically in Figure 8.1.
- → ¹³⁵Xe is produced to some degree directly in fission, but mostly as the result of the beta decay of its precursor ¹³⁵I (which has a half-life of 6.585 hours).



- ◆ ¹³⁵Xe is destroyed in two ways:
 - * through its own radioactive decay (135Xe has a half-life of 9.169 hours), and by absorption of neutrons to form 136Xe,
 - * ¹³⁵I is a direct product of fission, but can also appear through the radioactive decay chain ¹³⁵Te to ¹³⁵Sb to ¹³⁵I.
- ¹³⁵Te and ¹³⁵Sb have half-lives which are very short (19.0 s and 1.71 s) compared to those of ¹³⁵I and of ¹³⁵Xe;
- it is sufficient to model the decay of ¹³⁵Te and ¹³⁵Sb as "instantaneous", and add their fission yields to that of ¹³⁵I.



• The ¹³⁵Xe/¹³⁵I kinetics in any particular fuel bundle can thus be represented by the following equations:

$$\frac{dI}{dt} = \gamma_i \; \hat{\Sigma}_f \; \hat{\phi}_F \; - \lambda_i I \tag{8.1}$$

$$\frac{dX}{dt} = \gamma_x \, \hat{\Sigma}_f \, \hat{\phi}_F + \lambda_i I - \lambda_x X - \hat{\sigma}_X X \, \hat{\phi}_F \qquad (8.2)$$

• where:



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X = avge. conc. of <sup>135</sup>Xe in the bundle (atoms cm<sup>-3)</sup>
I = avge. conc. of ^{135}I in the bundle (atoms cm<sup>-3)</sup>
\gamma_{\rm v} = direct yield of <sup>135</sup>Xe per fission
\gamma_{\rm I} = direct yield of <sup>135</sup>I in fission, including yields of
                  135Te and 135Sh
\lambda_{\rm v} = decay constant of <sup>135</sup>Xe in s<sup>-1</sup>
\lambda_{\rm I} = decay constant of <sup>135</sup>I in s<sup>-1</sup>
 \hat{\phi} = bundle-average flux in the fuel (n.cm<sup>-2</sup>s<sup>-1</sup>)
 \hat{\Sigma}_f = macroscopic fission cross section of the fuel (cm<sup>-1</sup>)
and
 \hat{\sigma}_x = microscopic <sup>135</sup>Xe cross section (cm<sup>2</sup>)
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- In the above equations, the term $\gamma_i \hat{\Sigma}_f \hat{\phi}_F$ gives the ¹³⁵I production rate,
- while $\lambda_i I$ gives the ¹³⁵I loss rate (and the production rate of ¹³⁵Xe by iodine decay).
- Similarly, the term $\gamma_x \hat{\Sigma}_f \hat{\phi}_F$ gives the production rate of ¹³⁵Xe due to direct fission, while $\lambda_x X$ gives its decay rate.
- The term $\hat{\sigma}_x X \hat{\phi}_F$ represents the "destruction" (burnout) rate of ¹³⁵Xe due to neutron capture.
- Because of the comparable magnitudes of the various terms, the ¹³⁵Xe concentration is very sensitive to changes in flux level.



- The large absorption cross section of ¹³⁵Xe plays significant role in the overall neutron balance in the reactor,
- and thus directly affects the system reactivity, both in steady state and in transients.
- ◆ The ¹³⁵Xe/¹³⁵I kinetics also influences the spatial power distribution in the reactor.



8.2 Reactor Startup

- When a reactor is first started, or restarted after a long shutdown, the ¹³⁵Xe concentration will build up in all fuel bundles according to the equations derived above.
- In Figure 8.2, the variation of system reactivity as a function of time following startup is given for different final steady-state power levels.
- ◆ It can be seen that it takes ~ 40 hours for the ¹³⁵Xe concentration to fully reach equilibrium.



8.3 Steady-State Xenon Load

- At steady state the time derivatives dI/dt and dX/dt are zero.
- The above equations can then be solved to give the steady state concentrations of ^{135}I and ^{135}Xe (I_{ss} and X_{ss}):

$$I_{ss} = \frac{\gamma_i \hat{\Sigma}_f \hat{\phi}_F}{\lambda_i}$$
 (8.3)

$$X_{ss} = \frac{\left(\gamma_i + \gamma_x\right) \hat{\Sigma}_f \hat{\phi}_F}{\lambda_x + \hat{\sigma}_x \hat{\phi}_F}$$
 (8.4)



8.3 Steady-State Xenon Load

- It is obvious from these equations that,
- as a function of an increasing fuel flux ,
- the steady-state concentration of ¹³⁵I increases indefinitely,
- while in contrast that of 135 Xe tends to an asymptotic value which will be denoted $X_{ss,\infty}$:

$$X_{ss,\infty} = \frac{\left(\gamma_i + \gamma_x\right)\hat{\Sigma}_f}{\hat{\sigma}_v}$$
 (8.5)

• This asymptotic nature of the variation of X_{ss} with $\hat{\phi}_F$ is the reason why ^{135}Xe is termed a "saturating" fission product. (Other saturating fission products are ^{105}Rh , ^{149}Sm , ^{151}Sm , etc.)



8.3 Steady-State Xenon Load

- ◆ The limiting ¹³⁵Xe absorption rate at very high flux levels leads to a maximum reactivity of ~ -30 mk.
- In CANDU the equilibrium xenon load is approximately -28 mk.
- The flux level at full power in CANDU is such that the ¹³⁵Xe concentration is about 95% saturated, i.e., the average ¹³⁵Xe concentration is equal to about 95% of the value in an infinite flux.
- However, the steady-state ¹³⁵Xe concentration is not uniform in the core, but varies with flux according to Eq. (8.4). This is discussed in greater detail in Section 8.6 below.



- Due to the presence of the term $\sigma_{\mathbf{x}}\mathbf{X}\,\widehat{\phi_{F}}$,
- the variation of the ¹³⁵Xe concentration with flux is non-linear.
- The ¹³⁵Xe reactivity following power (flux) changes will depend on:
 - * the starting power level,
 - the time at that level,
 - the new power level, and
 - * the time spent at the new power level.



- Generally speaking, when the power is reduced from a steady level, the ¹³⁵Xe concentration increases at first.
- ◆ This is due to the fact that ¹³⁵Xe is still being produced by the decay of ¹³⁵I,
- but its burnout rate (by neutron absorption) is decreased because of the reduced neutron flux (reduced power).



- However, after a certain period (depending on the initial and final power and the rate of power reduction)
- the ¹³⁵I decay rate decreases sufficiently (due to the lower fission rate)
- that the rate of ¹³⁵Xe production drops below the rate of ¹³⁵Xe decay (and burnout).
- ◆ At this time, then, the ¹³⁵Xe concentration reaches a peak value and starts to decrease towards a new (lower) steady-state level.



- Conversely, when the power is increased from a steady level,
- the ¹³⁵Xe concentration will first decrease,
- and then go through a minimum
- and start increasing again to a higher steady-state level.



- Fig. 8.3 shows some typical reactivity variations due to ¹³⁵Xe following step reductions in power.
- Very similar variations, but in the opposite direction, ensue upon step increases in power.
- The quantitative effects will be different at different points in the core, due to the initial non-uniform distribution of ¹³⁵Xe.
- Thus, for an accurate assessment of xenon transients on the power distribution, a point-kinetics treatment is generally inadequate, and calculations in three dimensions will be required.



8.5 Xenon Transient Following a Shutdown

- Following a reactor shutdown, the burnout of ¹³⁵Xe stops,
- whereas the production by means of ¹³⁵I decay continues for several hours.
- The net result is that there is an initial increase in ¹³⁵Xe concentration and a decrease in core reactivity.
- If the reactor is required to be started up shortly after shutdown, extra positive reactivity must be supplied.
- The ¹³⁵Xe growth and decay following a shutdown for a typical CANDU is shown in Figure 8.4.



8.5 Xenon Transient Following a Shutdown

- It can be seen that, at about 10 hours after shutdown,
- the reactivity worth of ¹³⁵Xe increases to several times its equilibrium full-power value.
- ◆ At ~35-40 hours the ¹³⁵Xe has decayed back to its pre-shutdown level.
- If it were not possible to add positive reactivity during this period, every shutdown would necessarily last some 40 hours, when the reactor would again reach criticality.



8.5 Xenon Transient Following a Shutdown

- To achieve xenon "override" and permit power recovery following a shutdown (or reduction in reactor power), the CANDU-6 adjuster rods are withdrawn to provide positive reactivity.
- It is not possible to provide "complete" xenon override capability, this would require > 100 mk of positive reactivity.
- The CANDU-6 adjuster rods provide approximately 15 milli-k of reactivity, which is sufficient for about 30 minutes of xenon override following a shutdown.



- Xenon plays a role in the core power distribution.
- First, on the steady-state power distribution:
- → Because the steady-state ¹³⁵Xe concentration depends on the flux (Eq. 8.2),
- high-power bundles will have a higher xenon load, and therefore a lower reactivity, than low-power bundles of the same irradiation.

(cont'd)



- The effect of xenon is therefore to flatten the steadystate power distribution:
- The reduction in the maximum bundle power due to the local ¹³⁵Xe concentration can be of the order of 5%.
- ◆ For maximum channel power, ~3%.
- Thus xenon helps to increase the margin between peak powers and licensed limits.
- The xenon effect should be taken into account when accurate calculations are desired.



- The xenon effect also plays a role after refuelling:
- Fresh bundles introduced into the reactor have no xenon (or other fission products) at first.
- Fig. 8.2 shows it takes a day or so for the xenon to build to near saturation.
- Thus, the power of these fresh bundles is high at first and subsequently decreases, by perhaps several per cent, to their equilibrium value.



- Also, bundles which are shifted along the refuelled channel experience a change in power.
- In these bundles, corresponding to the specific power change (decrease or increase),
- there is a xenon transient similar to those in Fig. 8.3 (or the "opposite").
- This effect is not limited to shifted bundles, but occurs in any bundle whose power is affected by the channel refuelling operation, e.g. bundles in neighbouring channels.



- Transient-xenon effects due to refuelling are perhaps not always taken into account in routine core-follow calculations,
- but need to be captured when it is desired to perform the most accurate simulations,
- and to properly monitor the margin between peak powers and licensed limits.



- The RFSP history-based local-parameter methodology can simulate
- the spatially non-uniform effects of saturatingfission-product kinetics
- for both Xe-I and other fission-product pairs (e.g. Sm-Pm)
- in both steady-state and transient situations,
- by recalculating the ¹³⁵Xe concentration individually in every bundle, and therefore the effect on the lattice properties and on the core power distribution.



Summary

- ¹³⁵Xe is an important poison, which makes itself felt at all times.
- Its spatial and time effects must be taken into account when accurate fuel-management calculations are required.